

國立清華大學命題紙

96 學年度 核子工程與科學 系(所) 甲(工程) 組碩士班入學考試

科目 核工原理 科目代碼 3103 共 2 頁第 1 頁 *請在【答案卷卡】內作答

1. 是非題 (共 20 分，每小題答對者得 2 分，不答者不得分，答錯者倒扣 1 分)

- a) In the low energy region of most nuclei, inelastic scattering cross-section varies inversely as the neutron speed.
- b) The mean-life of a radioactive nuclide is longer than its half-life
- c) If the energies of the particles are of Maxwellian distribution, then the average energy is twice as large as the most probable energy.
- d) The water in a PWR is maintained at a high pressure of approximately 15 MPa. (1MPa = 145 psi)
- e) Charged particles are generally referred to as directly ionizing radiation.
- f) A reactor that sustains a steady-state chain reaction must be a “critical” reactor.
- g) The thermal utilization for a heterogeneous system is smaller than for the equivalent homogeneous mixture.
- h) The negative reactivity for both xenon-135 and samarium-149 show buildup after reactor shutdown.
- i) Sea water contains about 0.3 ppm of uranium.
- j) Resonance absorption increases with increasing temperature.

2. (7 分)

A ^{25}Mg -nucleus is at the 1.61 MeV excited level. Assume it to be at rest, what kinetic energy is imparted to it by the emission of a γ -ray as it deexcites to the ground state. (Data: 1 a.m.u. = 931.5 MeV)

3. (18 分)

Briefly explain the following terms:

- a) nuclear Doppler effect
- b) self-shielding
- c) thermal disadvantage factor
- d) bremsstrahlung
- e) stopping power
- f) Bragg curve

國立清華大學 命題紙

96 學年度 核子工程與科學 系(所) 甲(工程) 組碩士班入學考試

科目 核工原理 科目代碼 3103 共 2 頁第 2 頁 *請在【答案卷卡】內作答

4. (15 分)

- What is the difference between (n,n) and (n,n')?
- What is the difference between material buckling and geometric buckling?
- What is the difference between thermal shield and biological shield?

5. (15 分)

Consider a bare, spherical, homogeneous reactor of radius R. Due to symmetry, the flux in this reactor is a function only of r (distance from the center), and the reactor equation is

$$\frac{1}{r^2} \frac{d}{dr} r^2 \frac{d\phi}{dr} + B^2 \phi = 0$$

- Determine the flux distribution in this critical reactor.
- If \sum_f = macroscopic fission cross-section,
 E_R = recoverable energy per fission,
 P = operating thermal power of the reactor;

determine the flux at the center of the reactor.

6. (15 分)

Consider a point source emitting S neutrons/sec in an infinite diffusion medium.

- Find the flux distribution in the medium.
- Find the probability density that a source neutron emitted will be absorbed in (r, r+dr) from the source.
- Prove $L^2 = 1/6 \overline{r^2}$ (L^2 is the diffusion area, $\overline{r^2}$ is mean square distance to absorption)

7. (10 分)

Consider the neutron cycle of a thermal nuclear reactor, use block diagram to explain the six-factor formula.